

# Creation of Electric Fields in Tokamak Plasmas

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*Dedicated to Professor Dieter Pfirsch on his 60th Birthday*

In conjunction with energy and particle fuelling, considerable mechanical or electromagnetic momenta are transferred to fusion plasmas, e.g. through neutral beam injection, limiter recycling, pellet injection and RF heating.

The build up of electric fields through charge separation in the momentum absorption process is estimated for somewhat simplified cases of neutral particle injection and recycling.

It is shown that for the parameters of large present-day tokamak devices remarkable electric field strengths of the order of  $5 \cdot 10^5$  Volt/m can be created both in the plasma regions of beam absorption and near limiters.

## 1. Introduction

Electric fields are usually not considered explicitly in tokamaks except for those induced by external coil currents. However, a number of asymmetries, anomalous flows and transport, could be, at least formally, attributed to the action of an  $E$ -field. In this paper several mechanisms for electric field creation are pointed out on the basis of illustrative classical physics mechanisms. The pictures are developed for the example of neutral beam injection in Sects. 2 to 8, and then transferred to a recycling plasma at a limiter in Section 9. Corresponding arguments could also be readily applied to the cases of pellet injection and to RF-heating.

## 2. Injected Neutral Beam

Some gross effects of neutral beam injection on tokamak plasmas have been addressed in a formal way before [e.g. 1, 2], often without justifying the basic assumptions of symmetry etc. Analyses based on more elementary pictures do not seem to be available in the literature, although the physics is more transparent, more subtle effects appear, and numerical estimates can be obtained quickly. On

the other hand, the questions of consistency and completeness may be more open.

To illustrate the essential features let us simplify strongly the tokamak plasma geometry to a slab model as depicted in Figure 1. A beam of neutral particles (with density  $n_0$  and velocity  $v_{0x}$ ) enters the plasma from the left, propagating perpendicularly to the magnetic field pointing the upwards in  $z$ -direction. The plasma parameters such as electron density  $n_e(x)$ , temperatures  $T_e(x)$ ,  $T_i(x)$  are assumed to vary slowly with  $x$  as compared with

$$r_g = v_{0x} m_0 c / e B_z. \quad (1)$$

For simplicity we also assume hydrogenic plasma and beam particles, i.e.  $Z = 1$ . In a volume element

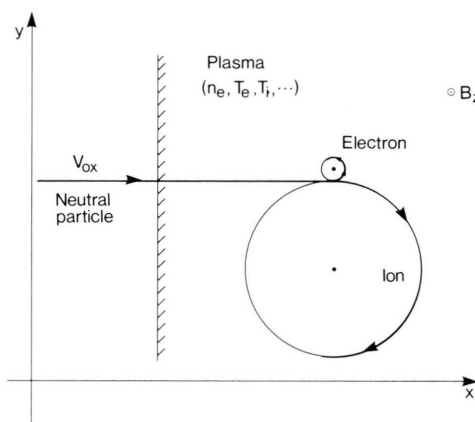


Fig. 1. Charge separation for absorbed neutral beam particle.

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inside the plasma, particles from the neutral beam are ionized at a rate

$$R_i = n_0 n_e \langle \sigma v \rangle_i, \quad (2)$$

where  $\langle \sigma v \rangle_i$  should include both electron and ion collisional ionization. The separated electron starts to gyrate with a speed corresponding to a few eV. The newly produced ion carries practically all the injection energy and moves into a relatively large orbit given by formula (1).

There is no fundamental difference if the beam is attenuated by charge exchange (cx) collisions at a rate

$$R_{cx} = n_0 n_i \langle \sigma v \rangle_{cx}. \quad (3)$$

The corresponding electron, then belongs to the background plasma around the collision point.

The separation of charges obviously creates an electric field oscillating with the local ion gyro-frequency. Averaged over such periods an electric dipole remains with a moment

$$p_y = -e r_g \quad (4)$$

with the radius  $r_g$  taken as the average distance between the charges.

It is, of course, not necessary that the electric field is built up between the ion and the original electron. In fact, certainly for

$$r_g > \lambda_D = \left( \frac{T_e T_i}{4 \pi n_e e^2 (T_e + T_i)} \right)^{1/2},$$

where  $\lambda_D$  is the Debye length, the plasma is adjusting on a plasma frequency timescale and the ion charge is shielded by other surrounding electrons; however, all adjustments must finally add up to the basic charge separation. In other words, the created field acts mainly on the background plasma until the fast particle is slowed down by collisions, i.e. for a slowing-down period which can be roughly estimated to be

$$\tau_{sd}(\text{sec}) \approx 10^{12} \frac{T_e(\text{keV})^{3/2}}{n_e(\text{cm}^{-3})}. \quad (5)$$

### 3. Polarization Model

To estimate the total electric field we might recall the elementary theory of polarization of dielectrics. There, an applied  $\mathbf{E}$ -field leads to the formation of

electric dipoles which give rise to a polarization field (in the counter direction). In our case the applied field is replaced by injection of particles. It should be noted that with fixed  $v_0$  and  $\mathbf{B}$  the created dipoles are all in the same direction, so that simple addition yields the total created field

$$E = 4 \pi e r_g \gamma N_f$$

with

$$N_f = (R_i + R_{cx}) \tau_{sd} \quad (6)$$

representing the total number of fast ions. The factor  $\gamma$  ( $\gamma < 1$ , e.g.  $\gamma = \frac{1}{2}$ ) could take into account that  $r_g$  is decreasing during a period  $\tau_{sd}$ .

If one is not willing to speculate on the lifetime of the fast ions, one could be satisfied with a creation rate (which corresponds to the initial displacement current)

$$\dot{E} = 4 \pi e r_g (R_i + R_{cx}). \quad (7)$$

An analogous consideration must be made for the magnetic field. Every beam particle absorption creates also a magnetic momentum connected with the gyrating fast ion:

$$\mu = \frac{e \omega_{ci}}{2 \pi c} r_g^2 \pi \quad (8)$$

directed opposite to the existing magnetic field ( $-z$  direction in Figure 1). The rate of change of the  $\mathbf{B}$ -field is given, in general, by

$$\dot{\mathbf{B}} = 4 \pi \mu \dot{N}_f = 4 \pi \mu R, \quad (9)$$

or for the above example by

$$\dot{B}_z = -2 \pi \frac{e}{c} \omega_{ci} r_g^2 (R_i + R_{cx}) \quad (10)$$

which would result in a diamagnetic field depression of the order of

$$\Delta B_z = 4 \pi \mu N_f.$$

### 4. Momentum Balance

In another, more macroscopic, approach the rate  $\dot{E}$  may be obtained from conservation of momentum. Referring again to Fig. 1 it is obvious that with absorption of a beam particle, its mechanical momentum  $m_0 v_{0x}$  is lost, i.e. the beam loses momen-

tum at a rate

$$-\dot{M}_x = m_0 v_{0x} \cdot (R_i + R_{cx}). \quad (11)$$

This momentum must appear in the momentum of the electromagnetic field (in Gaussian units), i. e.

$$\frac{d}{dt} \left( n m v + \frac{1}{4\pi c} (\mathbf{E} \times \mathbf{B}) \right) = 0. \quad (12)$$

In the rate of change for the field,

$$\frac{1}{4\pi c} \left\{ \frac{d\mathbf{E}}{dt} \times \mathbf{B} + \mathbf{E} \times \frac{d\mathbf{B}}{dt} \right\},$$

the second term is obviously of higher order, but it might be assessed through (9).

For our special case (Fig. 1), (12) becomes

$$\dot{E}_y = \frac{4\pi(R_i + R_{cx})}{B_z} (\mu E_y + c m_0 v_{0x})$$

with the solution

$$E_y \approx \frac{4\pi c m_0 v_{0x} (R_i + R_{cx})}{B_z} t \quad (13)$$

for

$$B_z \gg \tau_{sd} 4\pi\mu(R_i + R_{cx}),$$

in agreement with (7).

## 5. Example

For a numerical example data related to the JET experiment are chosen:  $B_z = 30$  kG,  $n_e = n_{D^+} = 3 \times 10^{13} \text{ cm}^{-3}$ ,  $T_e = 3$  keV, deuterium plasma and deuterium beam.

In the beams, particles with 80 keV energy are injected, adding up to a power of 10 MW through a port area of  $100 \times 30 \text{ cm}^2$ . Evaluation of these data results in (rounded figures)

$$v_{0x} = 3 \times 10^8 \text{ (cm s}^{-1}\text{)}$$

and

$$n_0 = 9 \times 10^8 \text{ (cm}^{-3}\text{)}$$

(or a flux density  $\Gamma_{NB} \approx 2.5 \times 10^{17} \text{ cm}^{-2} \text{ s}^{-1}$ ).

The actual injection angle deviates by about  $20^\circ$  from the normal, the effect of which is (unrealistically) ignored here but reconsidered below. The gyration frequency  $f_{ci}$  and the radius  $r_g$ , (1), are computed to

$$f_{ci} = \omega_{ci}/2\pi = 2.3 \cdot 10^7 \text{ s}^{-1}$$

and

$$r_g = 2 \text{ cm}.$$

With atomic cross sections from [3] the ionic collisional ionization with  $\langle \sigma v \rangle \approx 4.5 \times 10^{-8} \text{ cm}^3 \text{ s}^{-1}$  is higher than ionization by electrons ( $\langle \sigma v \rangle \approx 2 \times 10^{-8} \text{ cm}^3 \text{ s}^{-1}$ ). Charge exchange is the dominant process ( $\langle \sigma v \rangle_{cx} \approx 6 \times 10^{-8} \text{ cm}^3 \text{ s}^{-1}$ ). Thus, the sum of the collision rates,  $R$ , is

$$R = R_i + R_{cx} \approx 3.5 \times 10^{15} \text{ (cm}^{-3} \text{ s}^{-1}\text{)}.$$

Inserting these numbers into formula (7) we obtain

$$\dot{E} = 4.2 \times 10^7 \text{ esu} = 1.3 \times 10^{12} \text{ V m}^{-1} \text{ s}^{-1}.$$

Estimating the slowing down time  $\tau_{sd}$ , (5), to

$$\tau_{sd} = 0.15 \text{ s},$$

the enormous electric field strength of

$$E = 2 \times 10^{11} \text{ V/m}$$

would be created.

The diamagnetic field depression, (10), would amount to

$$\dot{B}_z = 4.5 \cdot 10^3 \text{ G s}^{-1},$$

so that the condition for the solution (13) is not well fulfilled; this could also be seen from the result that during a period  $\tau_{sd}$  the density of fast ions would considerably exceed that of the background and thus would invalidate some of our assumptions.

In reality there exist, however, reducing effects, several of which are discussed in the following sections.

## 6. Background Plasma Response

First of all, the background plasma acts like a dielectric to an applied field with a dielectric constant

$$\varepsilon = 1 + 4\pi n m_i c^2 / B^2, \quad (14)$$

(in our example,  $\varepsilon = 1250$  for deuterium).

The effective electric field strength as seen by particles entering the considered volume element is then

$$E^* = (1/\varepsilon) E. \quad (15)$$

This holds, for example, for subsequent beam particles as well as for background particles, and

could produce a drift velocity

$$\mathbf{w}_D = (c/B^2) (\mathbf{E}^* \times \mathbf{B}) \quad (16)$$

in the example of Fig. 1 in  $x$ -direction.

## 7. Oblique Beam Injection Motion along $\mathbf{B}$

As indicated above, real injection is not exactly perpendicular to  $\mathbf{B}$ . Therefore, the created fast ions move out of the considered volume element with a velocity component  $v_{\parallel}$ . If we assume, as for a tokamak, the existence of closed magnetic (or drift) surfaces, the fast ions will not accumulate in their birth volume element but rather fill up a volume connected with the corresponding flux surface.

For the present estimates we replace the toroidal shell by a planar ( $y-z$ )-shell with periodic boundary conditions and length  $l_{\parallel} = 2\pi R_t \cdot q$  with the major torus radius  $R_t$ , and the “safety factor”  $q$  as the measure of the helical field line pitch. In a tokamak this corresponds to the case of a “rational surface” where the field lines close on themselves. Thus our estimates might, for example, be directly applicable for beam deposition inside the  $q=1$  surface.

Assuming the fast particles to spread quickly over such a shell, the quantity  $N_f$  from formula (6) onwards should be reduced by the ratio between the beam penetrated part(s)  $F_p$  and the total area of flux surface  $F_s$

$$N_f^* = N_f F_p / F_s. \quad (17)$$

For the above JET exemple this ratio is around  $5 \times 10^{-3}$  (because of double penetration).

Here we have inserted for  $R_t = 300$  cm, for the (approximately) elliptic plasma cross-section, the half axes  $a = 110$  cm,  $b = 170$  cm, and chosen a surface about half way into the plasma.

The longitudinal motion effects, in principle, also the rates  $\dot{E}$  because a continuity equation with a flux term now determines  $\dot{N}_f$ :

$$\partial N_f / \partial t = -\text{div} (v_0 N_f) + R - R_{sd}. \quad (18)$$

For the purely perpendicular injection case we approximated this equation by

$$dN_f/dt = R - N_f/\tau_{sd} \quad (19)$$

with the solution (for constant source  $R$  and slowing-down time  $\tau_{sd}$ )

$$N_f(t) = R \tau_{sd} (1 - \exp \{-1/\tau_{sd}\}) \quad (20)$$

and

$$\dot{N}_f = R \exp \{-1/\tau_{sd}\}.$$

To obtain an estimate for the flux term we observe that a particle with sufficiently large  $v_{\parallel}$  on a tokamak flux surface returns to his birth volume element after a period

$$\tau_{\parallel} = 2\pi R_t q / v_{\parallel}. \quad (21)$$

Dividing the flux shells into a beam penetrated part ( $R \neq 0$ ) and the rest ( $R = 0$ ) we consider the boundary surfaces  $S_1$  and  $S_2$  between both regions (see Figure 2).

The flux through these surfaces consists of

- (a) Circulating particles, simply passing through the volume element;
- (b) Particles on their first circulation round.

The first kind does not contribute to the divergence term in (16). For the second kind,  $n_2$ , we take  $\tau_{\parallel} \ll \tau_{sd}$ ,  $\Delta z \ll 2\pi R_t q$  and average over  $\Delta z$  to obtain:

$$n_2 = R \tau_{\parallel}, \quad (22)$$

which leads (outside  $\Delta z$ ) to an average density gradient

$$|\nabla n| = R \tau_{\parallel} / 2\pi q R_t. \quad (23)$$

Resolving the divergence term for constant  $v_{\parallel}$  one arrives at:

$$(i) \quad -\text{div} (v_{\parallel} N_f) = v_{\parallel} R (\Delta z) \tau_{\parallel} / (2\pi q R_t) = R (\Delta z) \quad (24)$$

for the volume outside  $\Delta z$  (where the local  $R$  vanishes) because  $n_2 = 0$  at  $S_2$  during  $\tau_{\parallel}$ , and therefore

$$\dot{N}_f = R (\Delta z) - R_{sd} \quad (25)$$

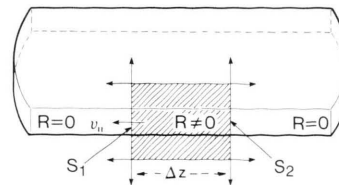


Fig. 2. Spreading of fast ions from the absorption region.

and

$$(ii) \quad -\operatorname{div}(v_{\parallel} N_f) = 0 \quad (26)$$

for the volume  $\Delta z$  because all particles  $n_2$  leaving, say, to the left are replaced by incoming ones from the right, and therefore

$$\dot{N}_f = R(\Delta z) - R_{sd}. \quad (27)$$

Thus, the same equation holds for the whole surface which justifies the above formula (17).

Equation (22), by the way, allows also an estimate of the toroidal asymmetry in the distribution of fast beam ions (using (6))

$$\Delta N_f / N_f = \tau_{\parallel} / \tau_{sd}. \quad (28)$$

For our example, with an injection angle of  $20^\circ$ ,  $v_{\perp} = 0.94 v_0$  and  $v_{\parallel} = 0.34 v_0$ , so that

$$v_{\parallel} = 10^8 \text{ cm s}^{-1}$$

and, for  $q = 2$ ,

$$\tau_{\parallel} = 4.10^{-5} \text{ sec}.$$

The asymmetry (28) is then

$$\Delta N_f / N_f = 2.10^{-4}.$$

## 8. Summary for Beam E-field

Finally, collecting all the above corrections we obtain for  $E^*$  the estimate

$$E^*(\psi) = \frac{4 \pi c m_i v_0 R \tau_{sd} F_p}{\varepsilon B_t F_s}, \quad (29)$$

which results, for our example, in the remarkable field strength of

$$E^* = 8 \cdot 10^5 \text{ V m}^{-1}.$$

The drift velocity which all plasma particles would experience in such a field is

$$v_{Dx} = 3 \cdot 10^7 \text{ cm s}^{-1}.$$

In reality, most quantities ( $R$ ,  $\tau_{sd}$ ,  $\varepsilon$ ,  $B_t$ ,  $F_p$ ,  $F_s$ ) in (27) are varying with the flux surface resulting in a  $E(\psi)$  profile. However, a drift velocity of such order would suggest a flattening of profiles throughout the main beam absorption region.

A more accurate evaluation with deposition profiles and plasma particle motion in proper

toroidal geometry can only be done numerically and is being prepared for a subsequent report. Here also the Pfirsch-Schlüter [4] neutralizing currents for more general  $q$ -values are discussed.

It is instructive to add some energy comparisons. In the electric field the energy density

$$E_{el} = \frac{1}{8 \pi} \varepsilon E^{*2} = 3.5 \times 10^4 \text{ erg cm}^{-3}$$

is stored, as compared to the kinetic energy density of the fast particles of

$$E_{kin} = \frac{m_D}{2} N_f^* v_{0x}^2 = 3.4 \times 10^5 \text{ erg cm}^{-3},$$

and the background thermal energy density of

$$E_{th} = 3 n k T \approx 4.3 \times 10^5 \text{ erg cm}^{-3}.$$

The energy  $E_{el}$  must be provided out of  $E_{fast}$ ; obviously it would lead only to a minor reduction in  $v_{0x}$  ( $\approx 10\%$ ) and a corresponding correction has been neglected here in view of, for example, the choice of  $\gamma = 1$  in formula (6). However,  $E_{kin}$  is rather close to  $E_{th}$ .

## 9. Recycling Boundary Plasma

Many of the above deliberations can be applied to plasma recycling. As indicated in Fig. 3, the plasma streams predominantly parallel to  $B$ , onto the limiter, is neutralized there, and the neutrals are said to be back-scattered into the plasma volume roughly with a cosine angular distribution around the limiter surface normal.

The incoming neutrals assume, through various atomic processes, velocities corresponding to a few, say 3, electrons volts; the density, however, must in steady state be comparable to that of the plasma in order to match the outflow.

As for the injected beam, the recycled neutral atoms are ionized or undergo charge exchange and their (perpendicular to  $B$ ) momentum is transferred.

For a typical tokamak boundary plasma:

$$\begin{aligned} n_e = n_{D^+} &= 10^{13} \text{ cm}^{-3}, & T_e = T_i &= 100 \text{ eV}, \\ n_0 &= 5 \cdot 10^{12} \text{ cm}^{-3}, & T_0 &= 3 \text{ eV}, & B &= 3 \cdot 10^4 \text{ G}, \\ \text{with } v_{0x} &= 2 \cdot 10^6 \text{ cm s}^{-1} \end{aligned}$$

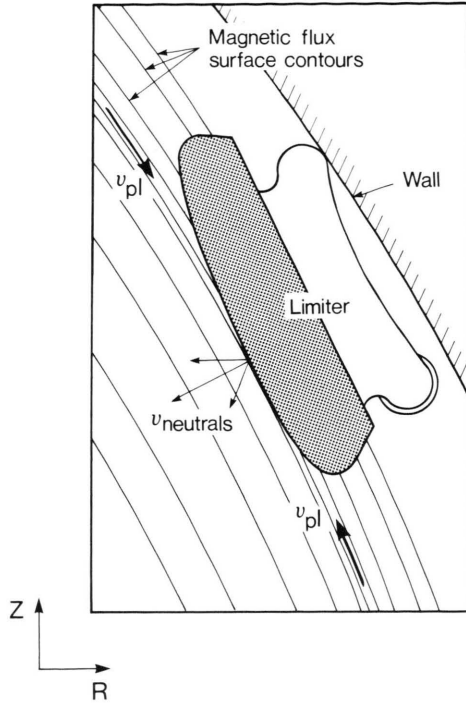


Fig. 3. Recycling of neutrals from a limiter surface.

we estimate the produced electric field strength with formulas directly corresponding to the ones developed for the beam case.

The slowing-down time, (5), must be replaced here by the collision time between ions

$$\tau_i(s) = 2.1 \cdot 10^7 \cdot T_i(\text{eV})^{3/2} \left( \frac{m_i}{m_H} \right)^{1/2} (n(\text{cm}^{-3}) \ln A),$$

which results for the above values in

$$\tau_i = 2 \times 10^{-4} \text{ s}.$$

The particle source  $R$  is estimated to be  $R \approx 4 \cdot 10^{+18} \text{ cm}^{-3} \text{ s}^{-1}$ , and the dielectric constant has dropped to  $\varepsilon \approx 430$ . The momentum balance, (13), then yields

$$\dot{E}_y \approx 3.4 \cdot 10^8 \text{ esu},$$

and

$$E = 2 \times 10^9 \text{ V/m}.$$

Corrected for background response ( $\varepsilon$ ) and the ratio between the surface of a belt limiter and the outermost magnetic flux surface, of about  $10^{-1}$  for the JET limiter case, we obtain in analogy to (29):

$$E^* = \frac{4 \pi c m_i v_0 R \tau_i F_{\text{lim}}}{\varepsilon B_i F_s}. \quad (30)$$

Inserting numbers in this formula we obtain

$$E^* = 4.7 \times 10^5 \text{ V m}^{-1}.$$

which is rather comparable to the  $E^*$ -value in Section 8.

We have assumed here again an almost homogeneous outer plasma layer. The mean free path  $v_{0x} \tau_i$ , however, indicates a more localized phenomenon, reducing  $F_s$  in (30).

Also this case will be considered in more detail in a future report for which a two-dimensional computational fluid plasma in realistic geometry is evaluated.

#### Acknowledgements

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[1] J. D. Callen, R. J. Colchin, R. H. Fowler, D. G. McAlees, and J. A. Rome, Proc. 5th IAEA Conf. on Plasma Physics and Contr. Nucl. Fusion Research, Tokyo 1974, Vol. I, p. 645.

[2] J. W. Connor and J. G. Cordey, Nucl. Fusion **14**, 185 (1974).

[3] A. C. Riviere, Nucl. Fusion **11**, 363 (1971).

[4] D. Pfirsch and A. Schlüter, Report MPI/PA/7/62.